Introduction

^{99m}Tc is extremely important for medical practice, and efforts to develop the best procedure for the production of its parent radionuclide, ⁹⁹Mo, for its subsequent use in technetium generators are being made for more than four decades. For this purpose, a composite ceramic target containing HEU dioxide and a filler metallic powder (AI, Mg, etc.) is irradiated in a high-flux reactor to obtain concentrate solution for fission Mo recovery by sorption, extraction or precipitation. However, this way has been restricted by IAEA because of nuclear nonproliferation and one of the options is to use irradiation of LEU dioxide without filler in commercial-level fluxes and to rise the concentration capability.

In this way extraction of Mo, U and certain FP from HNO₃ solutions with 0.2% solutions of higher hydroxamic acids (HA) in alcohols poorly soluble in water has been studied for ⁹⁹Mo concentrate production from solutions of different enriched U targets, including those of very low enrichment (3% ²³⁵U dioxide).

Experimental

Nitric acid solutions of molybdenum were prepared by dissolution of metallic Mo powder, and/or an acidified ⁹⁹Mo stock solution was used (specific activity 20 GBq_{*}g⁻¹ Mo). The activity concentration of ¹²⁵I stock solution was 4.5 GBq_{*}L-1. Experiments on the extraction of Mo with HAs were performed in the following concentration ranges, M: Mo, from 1.25 to 5.7; HAs, from 9.4 to 27. These concentration ranges were varied depending on the experimental conditions. Extraction experiments were performed in 15- and 50-mL test tubes. The phases were agitated for 1 min. with a Vortex laboratory vibration stirrer at a rotation rate of 3000 rpm, which was sufficient to attain the equilibrium.

The content of water-soluble hydroxamic acid in solutions was determined by spectrophotometry with Fe³⁺ in sulfuric acid. The content of metals in solutions was determined by ICP AES. The Mo concentration was determined by the standard colorimetric method with thiocyanate or by isotope dilution of ⁹⁹Mo with the measurement performed after the attainment of the radioactive equilibrium with ^{99m}Tc.

Results and discussion Mo extraction

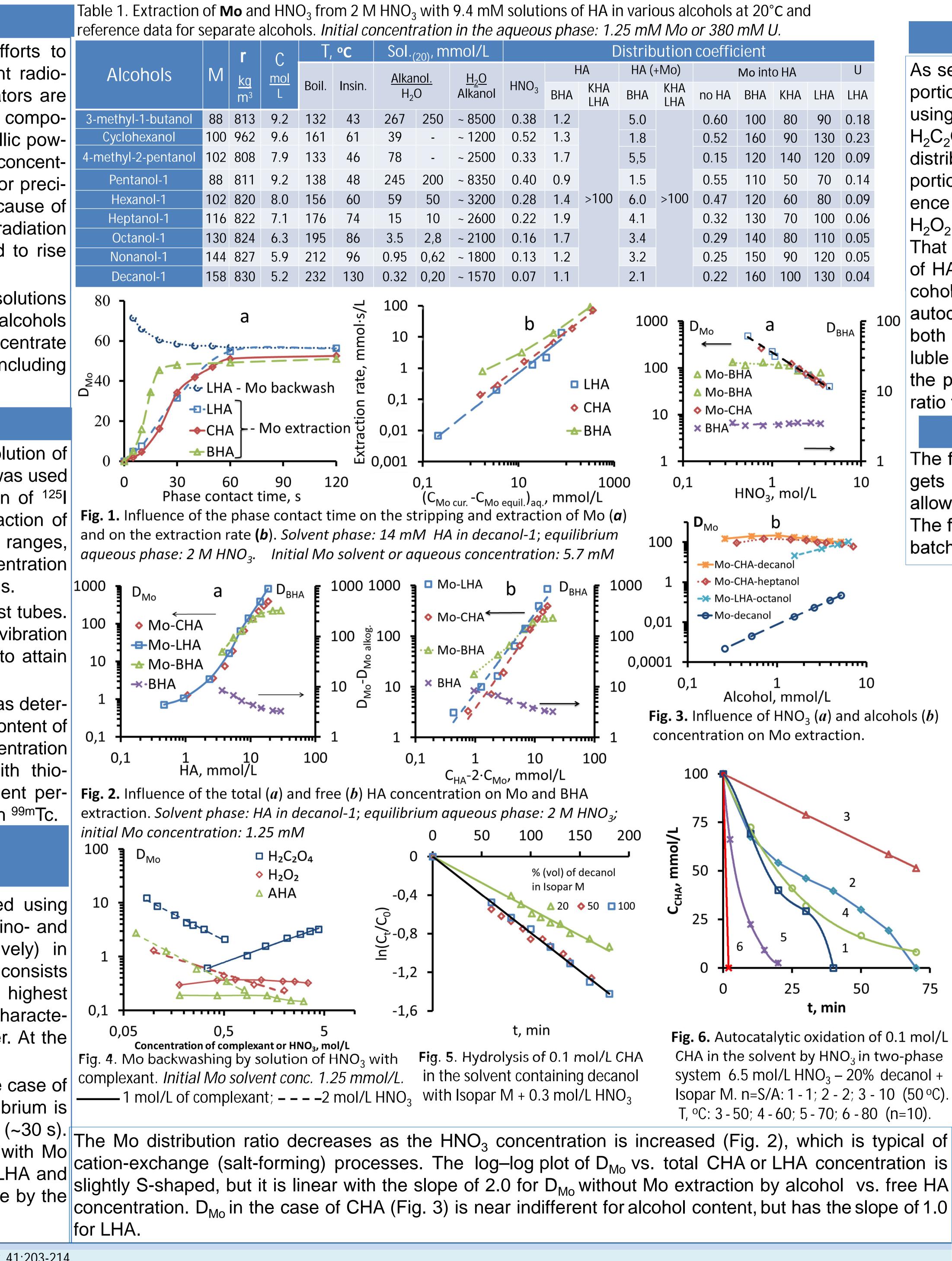
The extraction preconcentration of ⁹⁹Mo can be performed using solutions of poorly water-soluble HAs, namely benzo-, caprino- and laurylohydroxamic acids (BHA, CHA and LHA, respectively) in normal or branched alcohols C_5 - C_{10} . The optimum solvent consists of CHA in *n*-decanol (see Table 1), because it ensures the highest distribution ratio of Mo among the alcohols tested and is characterized by the highest flash point and by low solubility in water. At the same time, the extraction of U is insignificant.

Dynamics of the Mo mass transfer shows (Fig. 1) that, in the case of LHA and CHA, which are poorly soluble in water, the equilibrium is reached later (in 60 s) than in the case of water-soluble BHA (~30 s). Presumably, BHA forms in the aqueous phase a complex with Mo which then passes into the organic phase. In the case of LHA and CHA, the formation of a complex occurs only at the interface by the first order reaction. References

Zykov MP, Kodina GE (1999) Methods for Production of 99Mo (Review). Radiochemistry. 41:203-214. Goletskiy ND, Zilberman BYa, Kudinov AS, Voroshilov YuA, Ermolin VS, Yakovlev NG (2015). Development and Trials of a Process for Extraction Recovery of ⁹⁹Mo for Medical Purposes from Dissolved Irradiated Uranium Targets. Radiochemistry. 57:292–306. Naumov AA, Goletskiy ND, Zilberman BYa, Kudinov AS, and Murzin AA (2016). Possibilities of Concentrating ⁹⁹Mo by Extraction with Solutions of Aliphatic Hydroxamic Acids in Alcohols. Radiochemistry. 58:394–404. Naumov AA, Goletskiy ND, Zilberman BYa, and Murzin AA (2017). Peculiarities of Hydroxamic acid decomposition in nitric acid two-phase solutions containing alcohols and TBP as applied to molybdenum-99 backwashing. Radiochemistry. 59(6): (in printing). . Tkac P, Brown MA, Momen A, Wardle KE, Copple JM, Vandegrift GF (2017). MOEX – Solvent Extraction Approach for Recycling Enriched ⁹⁸Mo/¹⁰⁰Mo Material. Separ. Sci. & Technol. On-line pub. 20/03/2017 : 1-8

Concentrating of fission ⁹⁹Mo from very low enriched uranium by extraction with higher hydroxamic acids solutions in alcohols C_8 - C_{10} .

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The flowsheet testing included dissolution of U-AI model or real targets in 8 mol/L HNO₃ containing 0.2 g/L Hg and 0.2 g/L HF at 95 °C, allowing further I_2 and Ru compounds air stripping in the special tests. The further concentrating process was tested in counter-current and batch variants and the latter (Fig. 7) was found to be rather effective.

Solvent

to sorption Fig. 7. Flowsheet for ⁹⁹Mo separation from dissolved LEU target using caprinohydroxamic acid dissolved in 20% n-decanol + Isopar-M. (n = Solv/Aq by volumes, m = number of contacts).

Patents RU: 1. 2 522 544 (2014); 2. 2 575 028 (2016); 3. 2 624 920 (2017)

Fig. 6. Autocatalytic oxidation of 0.1 mol/L CHA in the solvent by HNO₃ in two-phase system 6.5 mol/L HNO₃ – 20% decanol + Isopar M. n=S/A: 1 - 1; 2 - 2; 3 - 10 (50 °C). T, °C: 3 - 50; 4 - 60; 5 - 70; 6 - 80 (n=10).





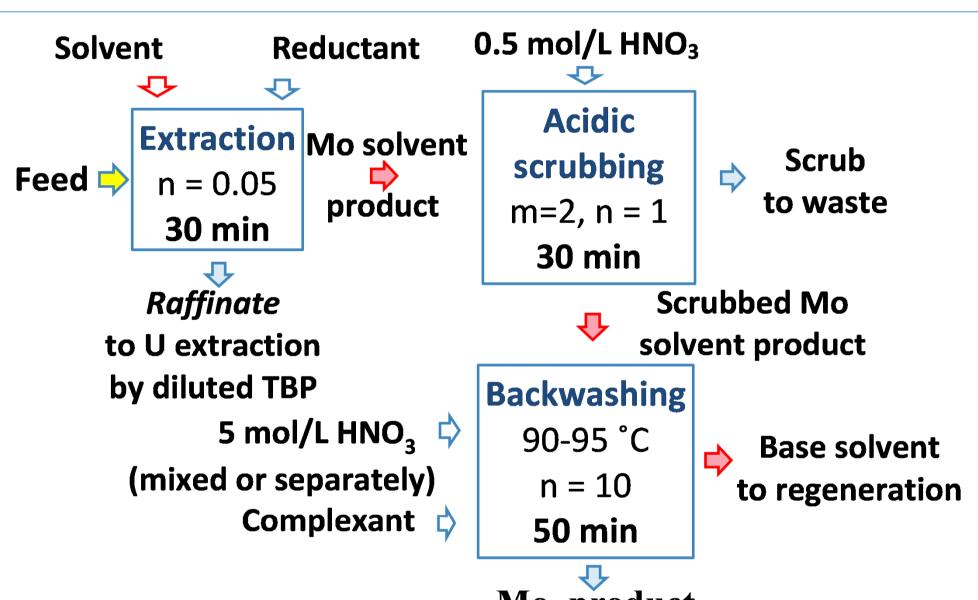
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Mo backwashing

As seen from Fig. 4, D_{Mo} distribution ratios decrease in inverse proportion to the square root of the ligand concentration in the case of using H_2O_2 and to the ligand concentration in the case of AHA and $H_2C_2O_4$. At the HNO₃ concentration increased over 1 mol/L the Mo distribution ratios in backwashing with AHA decrease in inverse proportion to the square root of the HNO₃ concentration, while the influence of the HNO₃ concentration on the Mo backwashing with the H_2O_2 solution is weak, and the total result is insufficient.

That is why HA decomposition has been tested as well. The main way of HA decomposition in HNO₃ solutions, as well as in equilibrium alcohol phase, is hydrolysis (Fig. 5). At the temperature above 50 °C autocatalytic HA decomposition takes place in the case of BHA in both phases, as well as at their mixing. In the case of low water-soluble CHA the autocatalysis is realized at 70-80 °C only (Fig. 6) during the phase mixing at the tendency to attenuation with n=S/A phase ratio from 1 to 10. It is practically useful to combine both methods.

Rig trials



Mo product

Mo extraction recovery was performed using 27 mmol/L CHA in 20% n-decanol with Isopar-M in 3 steps: extraction, scrubbing and backwashing - in the vessels of decreasing volume according to the concentrating factor. The simulate feed contained, mol/L: HNO₃ -1,2; AI - 1,2; Fe - 5 10⁻³; U - 0,11; Hg -1.10⁻³, ²³⁹Pu - 1,4.10⁻⁴, Mo ⁻ 3,2.10⁻⁵, as well as 15 MBq/L 99 Mo, 5,2 MBq/L 125 I and 4,1 MBq/L 239 Np.

The achieved total concentrating factor was 180 at process duration 2 h. Decontamination factors were $\sim 1.5 \cdot 10^6$ from U, ~ 850 from $^{125}I_{,} > 10^{5}$ from $^{239}Pu_{,} > 10^{6}$ from $^{239}Np_{,} > 10^{6}$ from AI, $4.6 \cdot 10^4$ from Fe, ~ 2•10⁴ from Hg.

The feasibility study has indicated that the compact extraction flowsheet and simple batch equipment are suitable for profitable ⁹⁹Mo recovery from LEU (standart 3-5% ²³⁵UO₂) targets. Final Mo decontamination for Tc generator production can be performed by sorption and/or by sublimation.

We assume the method to be an interlocutory decision between HEU irradiation and that of Mo isotopes ⁹⁸Mo or ¹⁰⁰Mo.